

The changes do not involve a significant reduction in the margin of safety, because the proposed changes affect only surveillance requirements, do not affect the function of the components and systems involved, and do not decrease the estimated equipment or system reliability.

Based on the NRC staff analysis, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room location:* University of Toledo Library, Documents Department, 2801 Bancroft Avenue, Toledo, Ohio 43606.

*Attorney for licensee:* Jay E. Silberg, Esquire, Shaw, Pittman, Potts and Trowbridge, 2300 N Street, N.W., Washington, DC 20037.

*NRC Project Director:* Leif J. Norrholm.

*Toledo Edison Company, Centerior Service Company, and The Cleveland Electric Illuminating Company, Docket No. 50-346, Davis-Besse Nuclear Power Station, Unit No. 1, Ottawa County, Ohio*

*Date of amendment request:* December 6, 1994.

*Description of amendment request:* The proposed amendment would revise Technical Specification (TS) 4.0.5, "Applicability" and its associated Bases; TS 3/4.1.2.3, "Reactivity Control Systems—Makeup Pump—Shutdown; TS 3/4.1.2.4, "Reactivity Control Systems—Makeup Pump—Operating; TS 3/4.1.2.6, Reactivity Control Systems—Boric Acid Pump—Shutdown; and TS 3/4.1.2.7, "Reactivity Control System—Boric Acid Pumps—Operating." The proposed change would replace the specific monthly surveillance requirements associated with the makeup pumps and boric acid pumps with a surveillance requirement referencing TS 4.0.5, which references Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code for quarterly pump testing requirements. The proposed change to TS 4.0.5 and its associated Bases would revise the requirement regarding the NRC's approval of relief requests to be in accordance with the NRC Staff's recommendation contained in NUREG-1482, "Guidelines for Inservice Testing at Nuclear Power Plants." Additionally, TS 4.0.5.a.2 which describes historical requirements for inservice inspection and testing would be deleted and TS 4.0.5.a.1 would be renumbered as TS 4.0.5.a.

*Basis for proposed no significant hazards consideration determination:*

As required by 10 CFR 50.91(a), the NRC Staff has performed an analysis of the issue of no significant hazards consideration, which is presented below:

(1) The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

Operation of the Davis-Besse Nuclear Power Station, Unit No. 1, in accordance with these changes, would not involve a significant increase in the probability of an accident previously evaluated because no accident initiators, conditions, or assumptions are affected by the proposed changes to replace the specific monthly surveillance requirements for the makeup and boric acid pumps with surveillance requirements referencing TS 4.0.5 (ASME Boiler and Pressure Vessel Code Section XI requirements) and to delete wording regarding NRC approval of relief requests. The changes do not involve a significant increase in the consequences of an accident previously evaluated, because no accident conditions or assumptions are affected that would increase the radiological consequences of a previously evaluated accident.

(2) The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not result in any new accident initiators nor do they alter any accident scenarios. The changes do not create the possibility of a different kind of accident from any accident previously evaluated, because the surveillance requirements for the makeup and boric acid pumps only affect the testing of existing components, systems, and functions, and do not introduce any new requirements.

(3) The proposed changes do not result in a significant reduction in the margin of safety.

The proposed changes do not reduce or adversely affect the capabilities or reliability of any plant structures, systems or components. Relaxation of the surveillance testing interval for the boric acid and makeup pumps and modifying the testing requirements is consistent with previous NRC guidance.

Based on this NRC staff evaluation, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Project Director:* Leif J. Norrholm.

*Union Electric Company, Docket No. 50-483, Callaway Plant, Unit 1, Callaway County, Missouri*

*Date of amendment request:* January 13, 1995.

*Description of amendment request:* The proposed amendment would revise Technical Specification (TS) by relocating Tables 3.3-2, "Reactor Trip System Instrumentation Response Times," and 3.3-5, "Engineered Safety Features Response Times," to FSAR Chapter 16, Section 16.3. The Bases discussion specific to Table 3.3-5 would also be relocated to FSAR Section 16.3.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The proposed revision does not involve a significant hazards consideration because operation of Callaway Plant with this change would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated.

Overall protection system performance will remain within the bounds of the accident analyses documented in FSAR Chapter 15, WCAP-10961-P, and WCAP-11883 since no changes to the response times or measurement interval are proposed.

The RTS and ESFAS will continue to function in a manner consistent with the above analysis assumptions and the plant design basis. As such, there will be no degradation in the performance of nor an increase in the number of challenges to equipment assumed to function during an accident situation.

These Technical Specification revisions do not involve any hardware changes nor do they affect the probability of any event initiators. There will be no change to normal plant operating parameters or accident mitigation capabilities. Therefore, there will be no increase in the probability or consequences of any accident occurring due to these changes.

(2) Create the possibility of a new or different kind of accident from any previously evaluated.