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Attorney for licensee: Jack Newman,
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Washington, DC 20036-5869.

NRC Project Director: Gail H. Marcus.

IES Utilities Inc., Docket No. 50-331,
Duane Arnold Energy Center, Linn
County, Iowa

Date of amendment request: March
28, 1995.

Description of amendment request:
The proposed amendment would revise
and clarify Technical Specification
Table 3.2-A that lists allowable out-of-
service times and surveillance test
intervals for instrumentation.

Basis for proposed no significant
hazards consideration determination:
As required by 10 CFR 50.91(a), the
licensee has provided its analysis of the
issue of no significant hazards
consideration, which is presented
below:

1. The proposed changes to TS Table 3.2-A will not significantly increase the probability or consequences of an accident previously evaluated. The changes do not alter the physical design or operation of the plant and serve to describe more accurately and clearly the actual logic configurations. The existing logic designs are in conformance with the Architect/Engineer's design documentation since plant startup. These changes will assure that the information in the tables is clearer and more consistent with the column headings of the table. The proposed changes do not affect assumptions contained in the plant safety analysis.

The Bases changes provide additional information about the logic arrangements as appropriate to identify unique or different logic configurations. Changes to the Allowed Outage Time (AOT) descriptions for the MSL Flow—High and MSL Tunnel Temperature—High provide clarification regarding application of the AOT to these logic arrangements, since multiple instrument channels provide input into multiple logic channels. This application conforms to the single failure criterion of the design basis (NEDO-10139, Compliance of Protection Systems to Industry Criteria: General Electric BWR Nuclear Steam Supply System, dated June 1970) and to the analytical basis for the TS (NEDC-31677P-A, Technical Specification Improvement Analysis for BWR Isolation Actuation Instrumentation, dated July 1990).

2. The proposed changes to Table 3.2-A will not introduce a new or different kind of accident from any accident previously evaluated. The changes do not alter the physical design of the plant or affect any modes of operation. The proposed changes serve to clarify the existing information to better assure that the trip instrumentation will be maintained as assumed in the accident analyses contained in the Updated Final Safety Analysis Report.

3. The proposed changes do not involve a significant reduction in a margin of safety. Clarification of the logic arrangements in both Table 3.2-A and the TS Bases and how the AOT is applied does not affect the ability of the isolation logic to perform its intended function. No physical changes to the plant are being made as part of this amendment.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Indiana Michigan Power Company,
Docket No. 50-315, Donald C. Cook
Nuclear Plant, Unit No. 1, Berrien
County, Michigan

Date of amendment request: March
17, 1995.

Description of amendment request:
The proposed amendment would defer
performance of the Type A containment
integrated leakage rate test until the next
refueling outage.

Basis for proposed no significant
hazards consideration determination:
As required by 10 CFR 50.91(a), the
licensee has provided its analysis of the
issue of no significant hazards
consideration, which is presented
below:

Per 10 CFR 50.92, a proposed change does not involve a significant hazards consideration if the change does not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated,
2. Create the possibility of a new or different kind of accident from any accident previously evaluated, or
3. Involve a significant reduction in a margin of safety.

Criterion 1

The Cook Nuclear Plant Type A test history provides substantial justification for the proposed test schedule. Three Type A tests were performed over a seven year period with successful results. The tests indicate that the Cook Nuclear Plant has a low leakage containment. In addition, there are no adverse trends in the results from the previous Types A, B, and C tests or visual inspections that indicate a gradual degradation of the containment boundary. Further, there are no structural modifications planned which would adversely affect the structural capability of the containment and that would be a factor in deferring the Type

A test one refueling outage. Containment leak rate testing is not an initiator of any accident. The proposed interval extension does not affect reactor operations or the accident analysis and has no radiological consequences, except for the dose savings associated with not performing the test. There will be no changes to 10 CFR 100 dose limits or the control room dose limits. Extending the test interval will not increase the probability of a malfunction of equipment important to safety. Based on these considerations, it is concluded that the change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Criterion 2

The proposed change does not involve physical changes to the plant or changes in plant operating configuration. The proposed change only relaxes the scheduler requirements for conducting one Type A test from the T/Ss and defers performance of the test one cycle. The purpose of the test is to provide periodic verification of the leak-tight integrity of the primary reactor containment, and systems and components which penetrate containment. The tests assure that leakage through containment and systems and components penetrating containment will not exceed the allowable leak rate values established in 10 CFR 50, Appendix J. Thus, it is concluded that the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Criterion 3

The proposed change to the schedule for performing the Type A test does not reduce the margin of safety assumed in the accident analysis for any release of radioactive materials or reduce any margin of safety preserved by the technical specifications. The methodology, acceptance criteria, and the technical specification leak rate limits for the performance of the Type A test will not change. Type A tests will continue to be performed in accordance with 10 CFR 50, Appendix J and the applicable Cook Nuclear Plant Technical Specifications beginning in 1997. In addition, there are no adverse trends in the results from the previous Type A, B, and C tests or visual inspections that indicate a gradual degradation of the containment boundary. Based on these considerations, it is concluded that the change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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